



Michael J. Yox
Regulatory Affairs Director
Vogtle 3 & 4

7825 River Road
Waynesboro, GA 30830
706-848-6459 tel

SEP 23 2021

Docket No.: 52-025

ND-21-0389
10 CFR 52.99(c)(1)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3
ITAAC Closure Notification on Completion of ITAAC 2.2.03.08c.vii [Index Number 192]

Ladies and Gentlemen:

In accordance with 10 CFR 52.99(c)(1), the purpose of this letter is to notify the Nuclear Regulatory Commission (NRC) of the completion of Vogtle Electric Generating Plant (VEGP) Unit 3 Inspections, Tests, Analyses, and Acceptance Criteria ITAAC item 2.2.03.08c.vii [Index Number 192]. This ITAAC performed an inspection of the as-built plate located above the containment recirculation screens and verified that the plate size and location satisfy the ITAAC acceptance criteria. The closure process for this ITAAC is based on the guidance described in NEI 08-01, "Industry Guideline for the ITAAC Closure Process under 10 CFR Part 52," which was endorsed by the NRC in Regulatory Guide 1.215.

This letter contains no new NRC regulatory commitments. Southern Nuclear Operating Company (SNC) requests NRC staff confirmation of this determination and publication of the required notice in the Federal Register per 10 CFR 52.99.

If there are any questions, please contact Kelli A. Roberts at 706-848-6991.

Respectfully submitted,

A handwritten signature in black ink, appearing to read "Michael J. Yox".

Michael J. Yox
Regulatory Affairs Director Vogtle 3 & 4

Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3
Completion of ITAAC 2.2.03.08c.vii [Index Number 192]

MJY/JFV/sfr

To:

Southern Nuclear Operating Company/ Georgia Power Company

Mr. Peter P. Sena III

Mr. D. L. McKinney

Mr. H. Nieh

Mr. M. D. Meier

Mr. G. Chick

Mr. S. Stimac

Mr. P. Martino

Mr. M. J. Yox

Mr. A. S. Parton

Ms. K. A. Roberts

Ms. J.M. Coleman

Mr. C. T. Defnall

Mr. C. E. Morrow

Mr. K. J. Drudy

Mr. J. M. Fisher

Mr. R. L. Beilke

Mr. S. Leighty

Ms. A. C. Chamberlain

Mr. J. C. Haswell

Document Services RTYPE: VND.LI.L06

File AR.01.02.06

cc:

Nuclear Regulatory Commission

Ms. M. Bailey

Mr. M. King

Mr. G. Bowman

Ms. A. Veil

Mr. C. P. Patel

Mr. G. J. Khouri

Mr. C. J. Even

Mr. B. J. Kemker

Ms. N. C. Coover

Mr. C. Welch

Mr. J. Gaslevic

Mr. O. Lopez-Santiago

Mr. G. Armstrong

Mr. M. Webb

Mr. T. Fredette

Mr. C. Santos

Mr. B. Davis

Mr. J. Vasquez

Mr. J. Eargle

Mr. E. Davidson

Mr. T. Fanelli

Ms. K. McCurry

Mr. J. Parent

Mr. B. Griman

Oglethorpe Power Corporation

Mr. R. B. Brinkman
Mr. E. Rasmussen

Municipal Electric Authority of Georgia

Mr. J. E. Fuller
Mr. S. M. Jackson

Dalton Utilities

Mr. T. Bundros

Westinghouse Electric Company, LLC

Dr. L. Oriani
Mr. D. C. Durham
Mr. M. M. Corletti
Mr. Z. S. Harper
Mr. J. L. Coward

Other

Mr. S. W. Kline, *Bechtel Power Corporation*
Ms. L. Matis, *Tetra Tech NUS, Inc.*
Dr. W. R. Jacobs, Jr., Ph.D., *GDS Associates, Inc.*
Mr. S. Roetger, *Georgia Public Service Commission*
Mr. R. L. Trokey, *Georgia Public Service Commission*
Mr. K. C. Greene, *Troutman Sanders*
Mr. S. Blanton, *Balch Bingham*

U.S. Nuclear Regulatory Commission
ND-21-0389 Enclosure
Page 1 of 3

Southern Nuclear Operating Company
ND-21-0389
Enclosure

Vogtle Electric Generating Plant (VEGP) Unit 3
Completion of ITAAC 2.2.03.08c.vii [Index Number 192]

ITAAC Statement

Design Commitment

8.c) The PXS provides RCS makeup, boration, and safety injection during design basis events.

Inspections, Tests, Analyses

vii) Inspection of the as-built components will be conducted for the plate located above the containment recirculation screens.

Acceptance Criteria

vii) The plate located above the containment recirculation screens is no more than 1 ft, 3 in above the top of the face of the screens and extends at least 8 ft, 3 in perpendicular to the front and at least 7 ft to the side of the face of the screens.

ITAAC Determination Basis

Multiple ITAAC are performed to demonstrate the Passive Core Cooling System (PXS) provides Reactor Coolant System (RCS) makeup, boration, and safety injection during design basis events. For this ITAAC, an inspection of the as-built components is conducted for the plate located above the containment recirculation screens (PXS-MY-Y02A and PXS-MY-Y02B) to confirm the plate is located no more than 1 ft, 3 in above the top of the face of the screens and extends at least 8 ft, 3 in perpendicular to the front and least 7 ft to the side of the face of the screens.

Inspections were performed of the as-built components by taking as-built measurements using survey equipment in accordance with Nuclear Construction and Startup Procedure (NCSP) 3-24, "Field Surveying" (Reference 1). NCSP 3-24 is used to establish layout and control points for determining the distance between the plate surfaces to the top and side face of the containment recirculation screens. The protective plate maximum height dimension is the distance between the top of the screens and the underside of the protective plate module top plate at the exposed edges of the protective plate, which extend into the containment recirculation water flow (east toward steam generator 2, and north toward the corridor), as described in the Updated Final Safety Analysis Report (UFSAR) Subsection 6.3.2.2.7 (Reference 2). Measurements are taken using survey equipment in accordance with site survey and measurement procedures to derive the distance between the as built components.

The results of the inspection are documented in the Unit 3 inspection report (Reference 3) confirming the plate is no more than 1 ft, 3 in above the top of the face of the screens (maximum as-built distance is 1 ft, 1 in), extends out at least 8 ft, 3 in perpendicular to the front of the screens (minimum as-built distance 8 ft, 7.6 in), extends at least 7 ft to the side of the face of the screens (minimum as-built distance is 7 ft, 0.3 in), which meets the ITAAC Acceptance Criteria.

Reference 3 is available for NRC inspection as part of the Unit 3 ITAAC 2.2.03.08c.vii Completion Package (Reference 4).

ITAAC Finding Review

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all ITAAC findings pertaining to the subject ITAAC and associated corrective actions. This review found that there are no relevant ITAAC findings associated with this ITAAC. The ITAAC completion review is documented in the ITAAC Completion Package for ITAAC 2.2.03.08c.vii (Reference 4) and is available for NRC review.

ITAAC Completion Statement

Based on the above information, SNC hereby notifies the NRC that ITAAC 2.2.03.08c.vii was performed for VEGP Unit 3 and that the prescribed acceptance criteria were met.

Systems, structures, and components verified as part of this ITAAC are being maintained in their as-designed, ITAAC compliant condition in accordance with approved plant programs and procedures.

References (available for NRC inspection)

1. Nuclear Construction and Startup Procedure (NCSP) 3-24, "Field Surveying"
2. VEGP 3&4 Updated Final Safety Analysis Report (UFSAR), Revision 10.0, Subsection 6.3.2.2.7 "IRWST and Containment Recirculation Screens"
3. SV3-PXS-M6K-800192, Rev. 0, "PXS ITAAC 192 Cont. Recirc. Screens ITAAC Inspections/Field Measurements"
4. 2.2.03.08c.vii-U3-CP, Rev. 0, ITAAC Completion Package